# Assessment of the Chernobyl Radiological Consequences

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### **INTRODUCTION**

The Chernobyl accident has been the worst accident in the history of peaceful use of nuclear energy. The nuclear explosion and the following 10 days graphite fire have not only completely destroyed the fourth unit's reactor of the Chernobyl NPP but, as well, resulted in the release of a large amount of radioactive species into the environment. This accident caused radioactive contamination in a lot countries of the Northern hemisphere, even in countries many thousand kilometers away from Chernobyl. For example, the deposition of different radionuclides has been found even in Japan [1-3].

The Belorussian Republic was affected by the Chernobyl accident much more than any other country in the world [4]. Twenty three percent of its territory were contaminated by caesium-137 with 1 Ci/km<sup>2</sup> and higher. As a result of the accident at the Chernobyl NPP, large territories in the Russian Federation and in the Ukraine have been contaminated as well [5-7].

Experts of different countries and international organizations are unanimous in the opinion that the Chernobyl accident has caused enormous socio-economic consequences in Belarus, the Russian Federation, and the Ukraine [4, 7-9]. It is also recognized that residents of the affected areas of the former USSR have been subject to various psychological burdens soon after the Chernobyl accident [4, 7-11].

However, there exists a significant controversy among specialists about the medical consequences (other than psychic) of the Chernobyl accident. For example, according to the assessment of J. Gofman [12] this accident will result in additional 475,000 fatal solid cancers, 19,500 leukemia and additional 475,000 non-fatal solid cancers. The abovementioned data were evaluated by J. Gofman [12] for all countries of the Northern hemisphere affected by the Chernobyl accident. Significant medical consequences, especially additional thyroid cancers and leukemia, have been as well predicted by E. Ivanov [13].

The whole International Radiation Community, on the contrary, rejected the possibility of serious medical consequences other than psychic stresses and feeling of anxiety suffered by the residents of the affected areas [8-11, 14-16]. Such controversy is unbelievable in the light of the reliable data established in the affected regions of the former USSR [4,7,17].

One of the reasons for such controversy has been explained by J. Gofman [18]. He managed to show that the abovementioned discrepancies in prognosis of medical consequences of the Chernobyl accident arose due to the incorrect risk coefficient of ionizing radiation employed by some specialists to forecast additional number of solid cancers and leukemia induced by Chernobyl radiation.

Another incorrectness that can seriously influence the forecasting of biomedical consequences of the Chernobyl accident is related to the collective dose estimation. It is well known, that assessments of authors [8, 12-16] have been made on the basis of data of nuclides release into the environment established soon after the accident [19, 20]. During the last years, however, more accurate data were established on the total discharge of radioactive species due to the Chernobyl accident, as well as more accurate data on deposition of different radionuclides in the affected areas of the former USSR [5-7].

The facts discussed above justify any independent analysis of possible medical consequences of the Chernobyl accident. This paper presents our attempt to assess the pure radiological consequences of the accident at the Chernobyl NPP.

The following limitations were taken into consideration in our assessment. First, we have restricted our analysis to stochastic effects of ionizing radiation only, such as solid cancers and leukemia. Second, we have also excluded from our analysis members of the personnel of the Chernobyl NPP and the cohort of liquidators. It means that our study was concentrated only on the population of areas affected by the Chernobyl accident. Third, our assessment was based on the use of simplified model of collective dose estimation very similar to the models used by authors [13-16,20]. The first step in collective dose assessment is the calculation of the exposition dose in air on the basis of experimental data on radionuclide deposition on the ground. The results of our assessment are given below.

# METHOD OF THE COLLECTIVE DOSE ASSESSMENT

The collective irradiation dose within some time  $\tau$ ,  $H_{\tau}^{coll}$ , can be estimated as:

$$H_{\tau}^{\text{coll}} = N \cdot \overline{H}_{\tau}$$
(1)

where,

$$\frac{N}{H_{\tau}} =$$
number of irradiated people,  
= mean individual effective equivalent  
dose accumulated within the time  **$\tau$** .

The value of  $H_{\tau}$  is estimated as a sum of external,

$$\overline{H}_{\tau, \text{ ext}}$$
, and internal,  $\overline{H}_{\tau, \text{ int}}$ , doses:  
 $\overline{H}_{\tau} = \overline{H}_{\tau, \text{ ext}} + \overline{H}_{\tau, \text{ int}}$  (2)

On the basis of data given in the Catalogue of doses [21] we have estimated that average contributions of external and internal irradiation to the equivalent doses delivered to inhabitants of the affected areas of Belarus in 1991-1992 were 60 and 40 percent respectively.

Assuming that the same contribution of external and internal radiation will be sustained at any time within a period of time  $\tau$  we can write:

$$\overline{H}_{\tau} = \frac{\overline{H}_{\tau,ext}}{0.6} , \qquad (3)$$

$$\overline{H}_{\tau} = \frac{\overline{H}_{\tau, \text{ int}}}{0.4} . \qquad (4)$$

Any of these expressions can be used for assessment of the collective equivalent dose delivered to the Belorussian population.

We shall base our investigation on the equation (3) because the procedure of assessment of  $H_{\tau,\text{ext}}$  is much easier than that of  $\overline{H}_{\tau, int}$ . The mean external equivalent dose,  $\overline{H}_{\tau,\,\mathrm{ext}}$  , can be determined by means of the equation:

$$\overline{H}_{\tau, \text{ ext}} = H^*_{\tau, \text{ ext}} \cdot \overline{A^0_s} (^{137} Cs)$$
 (5)

where

$$\overline{H}_{\tau, \text{ ext}}^* = \text{mean external individual} \\ \text{equivalent dose of inhabitants} \\ \text{living in areas with initial} \\ \text{contamination level by }^{137}\text{Cs equal} \\ \text{to 1 Ci/km}^2, \end{cases}$$

$$\overline{A_{s}^{0}}$$
 (<sup>137</sup>*Cs*) = average initial deposition level of  
<sup>137</sup> determined for the total area of  
contaminated territories.

Thus, the value of  $\overline{A_s^0}$  (<sup>137</sup> *Cs*) is given by:

$$\overline{A_{s}^{0}}(^{137}Cs) = \frac{Q_{0}}{S}$$
(6)

where

= total amount of  $^{137}$ Cs deposited on all  $Q_0$ affected territories (source term) S

= total area of contaminated territories

As can be seen from the equation given above the task of the collective equivalent dose assessment results in

values:  $\overline{H}_{\tau, \text{ ext}}^*$ ,  $Q_0$  and  $S_{\Sigma}$ . The mean individual equivalent dose of external irradiation normalized to the initial level of  $^{137}$ Cs contamination equal to 1 Ci/km<sup>2</sup>,  $H_{\tau, ind}^{*}$ , can be determined on the basis of measured or calculated exposition dose rates in contaminated areas. By means of exposition dose rates integration for the period of time  $\tau$  one can then compute the exposition dose  $P_{\tau}$ . This value can be transformed to the air-absorbed dose  $D_{\tau}$  by:

$$D_{\tau} = C_{\perp} \cdot P_{\tau} \tag{7}$$

where

 $C_1$  = conversion factor equal to 0.873  $\frac{rad}{R}$  [22]. By means of another conversion factor C<sub>2</sub> which is determined in  $\frac{rem}{rad}$ one can later estimate the required equivalent irradiation dose:

$$H_{\tau} = C_{2} \cdot D_{\tau} \tag{8}$$

# DOSE RATE AND DOSE IN AIR RECONSTRUCTION

There are only sparse measurement data on exposition dose rates in different areas of Belarus affected by the Chernobyl accident. Thus, one needs to assess these values by means of some computational methods and data on contamination of the soil.

The most simple model which can be used for computation of dose rates is based on the so-called model of infinite half-space source geometry at a reference height of 1 m. According to this model the exposition rate of external radiation at the time  $\mathbf{t}$  is given by:

$$R_{\gamma}(t) = \sum_{i=1}^{n} K_{i,\gamma} \cdot \sigma_{i}^{0} \cdot \exp\left(-\frac{0.693}{T_{\frac{1}{2}}^{i}} \cdot t\right) \cdot K_{i,L}(t)$$
(9)

where n

= number of nuclides contributing to the total exposition dose rate,

$$K_{i,\gamma}$$
 = conversion factor of *ith* nuclide per  
unit deposition, (mR/hr)/(Ci/km<sup>2</sup>),

$$\boldsymbol{\sigma}_{i}^{0} = \text{initial surface contamination with ith} \\ \text{nuclide normalized to initial surface} \\ \text{contamination equal to 1 Ci/km2 of 137Cs,} \\ \boldsymbol{T}_{1/2}^{i} = \text{half-life of ith nuclide,} \end{cases}$$

$$K_{i,L}$$
 (t) = shielding factor of *ith* nuclide originated from its penetration into the soil (function of time).

Computation of R (t) is usually carried out on the basis of the assumption, that all nuclides have the same shielding factor  $_{K \text{ L}}$  (t). Then the equation (9) can be written in the form:

$$R_{\gamma}(t) = K_{L}(t) \cdot \sum_{i=1}^{n} K_{i,\gamma} \cdot \sigma_{i}^{0} \cdot \exp\left(-\frac{0.693}{T_{j_{2}}^{i}} \cdot t\right),$$
(10)

Integration of the equation (10) from t=0 up to  $t=\tau$  gives the exposition dose of external radiation accumulated within the period of time  $\tau$ :

$$P_{\tau} = \int_{0}^{t} K_{L}(t) \cdot \sum_{i=1}^{n} K_{i,\gamma} \cdot \sigma_{i}^{0} \cdot \exp\left(-\frac{0.693}{T_{\frac{1}{2}}^{i}} \cdot t\right) dt.$$
(11)

Data on exposition dose rates and doses calculated on the basis of the equations (10) and (11) for different areas of Belarus are given in Fig. 1 and in Tables 1, 2 and 3. In computing these data experimental data on deposition of radionuclides as a result of the Chernobyl accident given in the book [5] were used (see Table 4). Table 5 consists of values of half-lives of gamma-emitting nuclides considered by

calculation of  $R_{\gamma}$  and  $P_{\tau}$ . The shielding factor  $K_{L}$  was calculated for the exponential model as it was done by T. Imanaka, T. Seo and H. Koide [23]:

$$K_{L} = a_{1} \cdot \exp(-l_{1} \cdot t) + a_{2} \cdot \exp(-l_{2} \cdot t),$$
 (12)

where

$$a_1$$
 and  $a_2$  = fractions of fast and slow  
migration of radionuclides into  
the soil

 $\underline{I}_1$  and  $\underline{I}_2$  = constants of vertical migrations into the soil

Values of these fractions and constants for different

periods of time after the Chernobyl accident are given in Table 6. They were determined from experimental data on vertical migration of radionuclides into the soil measured by specialists of Goshydromet of the former USSR [5]. They measured the vertical migration of radionuclides in the undisturbed soil for different kind of soils. We used data given in [5] for the sod-podsolic soil as more characteristic for contaminated territories of Belarus.

Figure 1 shows the change in exposition dose rates of external radiation in different areas of Belarus within the first weeks after the nuclear explosion at the Chernobyl NPP.

One can see that as a result of different composition or radionuclides deposited on the ground, roofs and walls of houses and other buildings the exposition dose rate in air in different contaminated areas of Belarus can differ by factor 3 even at the same <sup>137</sup>Cs contamination level.

The highest exposition dose rate had to be in the Northern sector of the so-called "nearest zone". This zone is a not very broad stripe that is extended in the Northern direction up to 100 kilometers from the Chernobyl NPP.

The lowest exposition dose rate had to be in the so-called "caesium spots", which are characterized by relative enrichment of radioactive deposition by caesium isotopes.

The difference in exposition dose rates diminished very quickly because of a radioactive decay of short-lived isotopes. At the end of the first month after the explosion at the Chernobyl NPP the exposition dose rates at different areas of Belarus were quite similar.



Fig. 1 Exposition dose rates in air at a height of 1 m during the first weeks after the Chernobyl accident in different areas of Belarus (normalized to the contamination level of caesium-137 equal to 1  $Ci/km^2$ ).

Table 1. Free air gamma exposition doses at a height of 1 m accumulated in different contaminated areas of Belarus within the period from 26.04.1986 to 26.04.2056 (normalized to the level of <sup>137</sup>Cs contamination of 1 Ci/km<sup>2</sup>, mR).

	X 7 . 1	Krasnopolje		Sector	Average
Year	Vetka rayon	rayon of	"Caesium spot"	"North" of the	composition of
	of Gomel oblast	Mogilev oblast	1	near zone	the total release
1986	258.0	212.7	225.1	622.5	369.1
1987	414.0	358.8	366.7	815.1	517.6
1988	531.7	471.9	476.8	947.9	628.0
1989	623.4	561.4	564.2	1048.6	715.5
1990	698.0	635.0	636.2	1128.8	787.5
1991	760.8	697.2	697.4	1195.1	848.7
1992	815.2	751.4	750.8	1251.7	902.1
1993	863.2	799.3	798.2	1301.3	949.5
1994	906.5	842.6	841.1	1345.7	992.4
1995	946.2	882.2	880.5	1386.1	1031.8
1996	983.0	919.1	917.2	1423.4	1068.5
1997	1017.4	953.4	951.4	1458.1	1102.7
1998	1049.8	985.8	983.8	1490.2	1135.0
1999	1080.5	1016.6	1014.4	1521.5	1165.7
2000	1109.9	1046.0	1043.8	1551.1	1195.1
2001	1138.0	1074.0	1071.8	1579.2	1223.1
2002	1164.9	1101.0	1098.7	1606.3	1250.0
2003	1191.1	1127.3	1125.1	1632.4	1276.3
2004	1216.3	1152.4	1150.1	1657.4	1301.4
2005	1240.3	1176.4	1174.1	1681.3	1325.4
2006	1263.2	1199.3	1197.1	1704.2	1348.3
2016	1447.8	1383.9	1381.6	1888.8	1532.8
2026	1573.8	1509.9	1507.6	2014.7	1658.7
2036	1660.8	1596.9	1594.6	2101.8	1745.7
2046	1721.1	1657.2	1654.9	2162.1	1806.0
2056	1760.4	1696.6	1694.3	2201.4	1845.3

 Table 2.
 Contribution of different radionuclides to the summary free air gamma exposition dose, accumulated within the period from 26.04.1986 to 31.08.1986 in contaminated areas of Belarus.

	D <sub>i</sub> / D, %									
Radionuclide	Vetka rayon of Gomel oblast	Krasnopolje rayon of Mogilev oblast	"Caesium spot"	Sector "North" of the near zone	Average composition of the total release					
<sup>239</sup> Np	0	0	0	0.9	0.8					
<sup>99</sup> Mo	0	0	0	0.2	0.1					
<sup>132</sup> Te	25.3	21.6	26.9	22.1	19.5					
$^{131}$ I	12.6	10.7	10.5	6.6	7.3					
<sup>140</sup> Ba	9.1	7.6	6.5	11.3	16.6					
<sup>141</sup> Ce	0	0	0	0.8	0.5					
<sup>103</sup> Ru	10.0	11.1	11.6	10.2	5.2					
<sup>95</sup> Zr	0	0	1.4	27.8	25.0					
<sup>110m</sup> Ag	0	0	0	0	0					
<sup>144</sup> Ce	0	0	0	0.7	0.7					
<sup>106</sup> Ru	6.8	3.3	4.0	3.7	2.0					
<sup>134</sup> Cs	21.1	26.7	22.1	9.9	12.6					
<sup>125</sup> Sb	0	0	0	0	0					
<sup>137</sup> Cs	15.1	19.0	17.0	5.8	9.7					

According to [7], there are places in the 30 kilometers zone with level of contamination by <sup>137</sup>Cs as high as many hundred Ci/km<sup>2</sup>. It means that soon after the explosion the exposition dose rate in air could reach in such places many hundred millirentgen per hour and therefore exceed many hundred thousand times the exposition dose rate in air before the Chernobyl accident.

According to our calculation nuclides with half-lives shorter than 1 year played an important role during the first months after the Chernobyl accident.

It can be seen from Table 3, where individual contributions to the total exposition dose of external radiation of different nuclides at different areas of Belarus are given.

Data presented in Table 2 show that short-lived

	D <sub>i</sub> / D, %							
Radionuclide	Vetka rayon	Krasnopolje rayon	"Cassium anot"	Sector "North" of				
	of Gomel oblast	of Mogilev oblast	Caesium spot	the near zone				
<sup>239</sup> Np	0	0	0	0.2				
<sup>99</sup> Mo	0	0	0	0				
<sup>132</sup> Te	2.7	1.9	2.5	5.1				
$^{131}$ I	1.3	0.9	1.0	1.5				
<sup>140</sup> Ba	1.0	0.7	0.6	2.6				
<sup>141</sup> Ce	0	0	0	0.2				
$^{103}$ Ru	1.2	1.1	1.2	2.6				
<sup>95</sup> Zr	0	0	0.2	8.3				
<sup>110m</sup> Ag	0	0	0	0				
<sup>144</sup> Ce	0	0	0	0.6				
<sup>106</sup> Ru	3.0	1.2	1.6	3.4				
<sup>134</sup> Cs	16.4	17.0	15.5	16.0				
<sup>125</sup> Sb	0	0	0	0				
<sup>137</sup> Cs	74.4	77.2	77.4	59.5				

 Table 3.
 Contribution of different radionuclides to the summary free air gamma exposition dose, accumulated within the period from 26.04.1986 to 31.04.2056 in contaminated areas of Belarus.

Table 4. Composition of nuclide deposition in the affected areas of Belarus (normalized to the activity of 137-Cs) [5]

			D <sub>i</sub> / D, %		
Radionuclide	Vetka rayon of Gomel oblast	Krasnopolje rayon of Mogilev oblast	"Caesium spot"	Sector "North" of the near zone	Average composition of the total release
<sup>239</sup> Np				33.33	6.67
<sup>99</sup> Mo				3.33	3.33
<sup>132</sup> Te	14.13	9.5	12.78	33.3	16.7
<sup>131</sup>	14.13	9.5	10.0	20.0	16.7
<sup>140</sup> Ba	1.086	0.714	0.67	3.67	3.33
<sup>141</sup> Ce				3.67	3.67
<sup>103</sup> Ru	1.957	1.714	1.994	5.33	2.67
<sup>95</sup> Zr			0.056	3.33	3.33
<sup>110m</sup> Ag					0.01
<sup>144</sup> Ce				2.00	2.33
<sup>106</sup> Ru	1.413	0.55	0.722	2.00	1.0
<sup>134</sup> Cs	0.545	0.545	0.5	0.67	0.5
<sup>125</sup> Sb					0.02
<sup>137</sup> Cs	1.00	1.00	1.00	1.00	1.0

nuclides determined more than 50 percent of the total exposition dose in air delivered within the first months after the accident.

In case of the Northern sector of the so-called "near zone" (Bragin district is settled in this zone) short-lived nuclides gave about 85 percent of the total exposition dose.

It is clear that one needs to consider the total spectrum of the deposited radionuclides in the assessment of the possibility of direct effects of radiation on inhabitants of 30 kilometers zone, that were evacuated within the first months after the Chernobyl accident.

The total contribution of short-lived nuclides to the exposition dose of external radiation which can be delivered within quite a long period is not very large. Such conclusion may be drawn on the basis of data given in Table 3. In this table results of the exposition doses calculated for different areas of Belarus for the period of time from 26.04.1986 to 26.04.2056 (70 years period) are presented.

As can be seen from this table nuclides with half-life shorter than the half-life of isotope  $^{134}$ Cs, give together only about 7% of the total exposition dose in air delivered during the 70 years after the Chernobyl accident.

It means that caesium isotopes determine practically the total exposition dose of external radiation delivered during a rather long period of time after the Chernobyl accident.

So, in the assessment of the total collective irradiation dose one does not have even to take into consideration nuclides other than caesium isotopes and concentrate the efforts on accuracy of

Nuclide	Half life, T <sub>1/2</sub> , days	Conversion* factor, K <sub>i,8</sub>
<sup>239</sup> Np	2.35	2.9
<sup>99</sup> Mo	2.73	5.1
<sup>132</sup> Te	3.27	45.9
131	8.04	7.3
<sup>140</sup> Ba	12.6	41.5
<sup>141</sup> Ce	32.5	1.2
<sup>103</sup> Ru	39	8.9
<sup>95</sup> Zr	64	28.3
<sup>110m</sup> Ag	250	50.3
<sup>144</sup> Ce	284	0.8
<sup>106</sup> Ru	368	3.8
<sup>134</sup> Cs	755.6	29.1
<sup>125</sup> Sb	985.5	7.9
<sup>137</sup> Cs	11023	10.7

 Table 5. Half lives and conversion factors of radionuclides deposited in the areas affected by the Chernobyl accident [5].

\* Note: Conversion factors are given in units

 $\frac{mR/H}{Ci/km^2}$ 

Table 6. Parameters of nuclide vertical migration in soil.

Period of time, years	<u>a</u> 1	$l_1$ year <sup>-1</sup>	<u>a</u> <sub>2</sub>	l <sub>2</sub> year <sup>-1</sup>
$0 \le t \le 1$ year	0.61	1.25	0.39	- 0.4
$0.5 \le t \le 16.5$ years	0.4407	0.18	0.4068	$2.75 \bullet 10^{-3}$
$16.5 \le t \le 100$ years	0.5345	0.18	0.4934	1.375•10 <sup>-2</sup>

computational models and accuracy of data on contamination levels of caesium isotopes.

Only in case of the Northern sector of the "near zone" the contribution of nuclides other than caesium isotopes to the total exposition dose in air delivered over the 70 after the Chernobyl accident has reached about 25%.

It is well known, that radioactive contamination of the soil in the major part of the affected territories of Belarus, the Russian Federation and the Ukraine is similar to contamination in "caesium spots".

This fact simplifies the assessment of collective irradiation doses of the affected population in Belarus, Russia and the Ukraine because it allows to use some uniform data that transform data on the initial surface contamination of the soil to external exposition doses.

According to the data given in Table 1, the total exposition dose of external radiation in air at a reference height of 1 m delivered over the 70 years after the Chernobyl accident in an area with <sup>137</sup>Cs isotope contamination equal to 1 Ci/km<sup>2</sup> is about 1,700 mR.

The contribution of the  $^{137}$ Cs isotope to the total exposition dose is about 1,311 mR or 77.4 percent within the 70 years period.

### DOSE RATES AND DOSES IN

## SETTLEMENTS ENVIRONMENT

It is clear that the data established for the areas with undisturbed soil can not be used for assessment of the collective irradiation doses delivered by external radiation because only a small fraction of population affected by the Chernobyl accident has a regular access to such areas.

For example it was established by the Ukrainian specialists [7] that children at the age of 7 years and under living in contaminated areas spend only 13 percent of the day outside the houses and other buildings. This means automatically that such children forming the most sensitive subpopulation to irradiation spend 87 percent of day inside the buildings and at least 87 percent of their time in settlements.

Children and teenagers at the age of 7-18 years according to the Ukrainian National Report [7] (see Table 4.1.3 on pages 4.18) spend about 18 percent of their time outside the buildings and this means about 82 percent inside the buildings.

The largest fraction of time outside the buildings spend pensioners. It reaches 36 percent of the day.

These data show that the inhabitants of contaminated areas spend about 20 percent of their time outside the buildings and 80 percent of time inside the buildings.

	uv/II/[#+]•						
Settlement	Mean level of contamination, Ci/km <sup>2</sup>	Street	Bench	Yard	Bed	Garden	House
Fedorovka	2.2	19	18	19	20	21	14
Glinnoe	5.2	27	23	27	28	28	15
Klintsy	5.9	20	19	19	21	22	14
Veliki Topol	7.1	28	28	27	30	32	16
Lopatni	8.1	32	30	31	34	34	17
Unoshevo	10.8	35	39	39	45	48	17
Lesnovka	15.9	54	51	49	61	73	26
Novo-zybkov	18.5	53	52	49	58	62	19
Gordeevka	21.6	50	65	67	79	84	25
Zlynka	28.4	106	85	90	87	100	25
Vyshkov	29.6	82	85	89	100	109	47
Mirnyi	32.6	58	65	78	109	113	22
Staryi Vyshkov	33.9	145	132	108	112	143	32

Table 7. Mean exposition dose rates in different areas of Bryansk oblast in autumn 1990 at a height of 1 m ( $\mu$ R/h) [24].

Table 8. Mean exposition dose rates outside and inside the buildings in contaminated areas of Bryansk oblast in autumn 1990 at a height of 1 m (µR/h).

	0		
Settlement	Mean level of contamination, Ci/km <sup>2</sup>	Outside the building	Inside the building
Fedorovka	2.2	19.4	14
Glinnoe	5.2	26.6	15
Klintsy	5.9	20.2	14
Veliki Topol	7.1	29	16
Lopatni	8.1	32.2	17
Unoshevo	10.8	41.2	17
Lesnovka	15.9	57.6	26
Novozybkov	18.5	54.8	19
Gordeevka	21.6	69	25
Zlynka	28.4	93.6	25
Vyshkov	29.6	93	47
Mirnyi	32.6	84.6	22
Staryi Vyshkov	33.9	128	32

These values show that at least 80 percent of time the inhabitants of contaminated areas spend in settlements or areas with disturbed soil.

Therefore, use of the data calculated for areas with undisturbed soil for the assessment of the collective dose will cause a significant overestimation of the collective doses.

In order to avoid this mistake one needs to correct the equations (10) and (11) by introducing an additional correcting factor  $K_S$ , which is determined as a ratio of the exposition dose rate in air measured in settlements to the exposition dose rate in air over the undisturbed soil measured at the same level of radioactive contamination.

The equations (10) and (11) can be then rewritten as follows:

$$R^*_{\gamma}(\mathsf{t}) = K_{s} \cdot K_{\scriptscriptstyle L}(\mathsf{t}) \sum_{i=1}^n K_{i,\gamma} \cdot \sigma^{\scriptscriptstyle 0}_{i} \cdot \exp \left(-\frac{0.693}{T^i_{j_2}} \cdot \mathsf{t}\right),$$

$$P_{\tau}^{*} = K_{s} \cdot \int_{0}^{\tau} K_{L}(t) \cdot \sum_{i=1}^{n} K_{i,\gamma} \cdot \sigma_{i}^{0} \cdot \exp(-\frac{0.693}{T_{j_{2}}^{i}} \cdot t)$$
(14)

where:

 $R_{\gamma}^{*}$  = exposition dose rate at a reference  $R_{\gamma}^{*}$  (height of 1 m in the settlements environment.

 $P_{\tau}^{*}$  = exposition dose of external radiation in air at a reference height of 1 m in the settlements environment delivered over a period of time  $\tau$ 

We describe here a very simple method that allows to consider the differences in external radiation in settlements and in areas with the undisturbed soil. It is based on experimental data on exposition dose rates measured by specialists of the scientific organization "Typhoon" of the Goshydromet of the former USSR

(13)

[24].

Specialists of this organization have carried out extensive measurements of dose rates in autumn 1990 in different contaminated settlements of the Bryansk oblast.

They performed measurements of exposition dose rates in the streets, yards, on benches, over beds of kitchen gardens, in gardens and inside the houses as well. Dose rates measured in the course of this investigation and averaged for each studied settlement are given in Table 7.

On the basis of data given in columns 3-7 of Table 7 we can estimate the mean arithmetic dose rates

outside the buildings,  $R_{\gamma}$  (t)<sub>out</sub>, by:

$$=\frac{R_{\gamma}(t)_{out}}{5}$$
(15)

where:

$$R \gamma (t)_{str.}$$
 = exposition dose rates measured in the streets,

- $R_{\gamma}(t)_{bn}$ . = exposition dose rates measured at benches,
- $R_{\gamma}(t)_{y}$  = exposition dose rates measured in yards,
- $\frac{1}{R_{\gamma}} (t)_{b} = \text{exposition dose rates measured} \\ \text{over beds of kitchen gardens,}$

$$R \gamma (t)_g$$
 = exposition dose rates measured in gardens.

Values of  $R_{\gamma}(t)_{out}$  determined this way are presented in the third column of Table 8. Here are also given the values of exposition dose rates measured by [24] inside the houses for comparison (see the fourth column of Table 8).

Values of  $R_{\gamma}(t)_{out}$  as a function of effective contamination levels, *Aeff*, are also shown in Fig. 2. Here *Aeff* is estimated by:

$$Aeff = \frac{1}{K_{\gamma}(^{137}Cs)} \cdot \sum_{i=1}^{n} K_{i,\gamma} \cdot A_{i}^{\circ} \cdot \exp(-\frac{0.693}{T_{j_{2}}^{i}}t),$$
(16)

where:

$$K_{\gamma} \begin{pmatrix} ^{137}Cs \end{pmatrix} = \text{conversion factor of } ^{137}Cs \text{ per unit} \\ \text{deposition, } (\mu R/h)/(Ci/km^2), \\ A_{i}^{\circ} = \text{initial surface contamination of the} \\ \text{soil by ith nuclide.} \end{cases}$$

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Figure 2 shows the existence of a strong dependence of  $R_{\gamma}(t)_{out}$  from the effective contamination levels *Aeff* that can be described on the basis of the linear function:

$$R_{\gamma}(t)_{\text{out}} = A + B \cdot Aeff,$$
 (17)

where *A* and *B* are some constants.

They have a very easy meaning. As can be seen from

(17), at  $Aeff \rightarrow 0 \ R \gamma$  (*t*)<sub>out</sub> $\rightarrow A$ . It means that *A* is the exposition dose rate on territory not contaminated by the Chernobyl accident or the exposition dose rate of the background radiation. The constant *B* in the equation (17) is the exposition dose rate in autumn 1990 per unit of the effective contamination Aeff, ( $\mu$ R/h)/(Ci/km<sup>2</sup>).

On the basis of the standard LSM procedure we have established the following values of A and B:  $A=8.555 \ \mu$ R/hr,  $B = 2.381 (\mu$ R/h)/ (Ci/km<sup>2</sup>).

Substitution of quantities A and B into the equation (18) transforms it to:

$$R_{\gamma}(t)_{\text{out}} = 8.555 + 2.381 \cdot Aeff,$$
 (18)

where  $R_{\gamma}(t)_{out}$  is expressed in  $\mu$ R/h.

The solid line in Fig. 2 was drawn on the basis of the equation (18).

We have performed the same procedure with the data on measured exposition dose rates established by [24] inside the houses of settlements of the Bryansk oblast (see column 8 of Table 7 or column 4 of Table 8).

For the exposition dose rates inside the buildings

 $R_{\gamma}(t)_{in}$ , we have established the following formula:

$$R_{\gamma}(t)_{\rm in} = 11.814 + 0.505 \cdot Aeff \tag{19}$$

The solid line in Fig. 3 was calculated by means of the equation (19). One needs to notice much larger scattering of measured data from the solid line in Fig. 3 in comparison to Fig. 2.

This phenomenon has an easy explanation. Conditions of irradiation outside the houses and other buildings in different settlements of the Bryansk oblast do not differ very much. On the contrary conditions of irradiation inside the houses and other buildings can differ very significantly. It is known that in rural settlement houses and other buildings are mostly constructed of wood that has much lower attenuation capability than brick or concrete used as a construction materials in urban settlements. Therefore, the exposition dose rates inside the urban buildings are lesser than in rural settlements even if the exposition dose rates outside the buildings are the same. This is a reason of much larger scattering of experimental points from solid line in Fig. 3 than in Fig. 2.



Fig. 2. Exposition gamma dose rates at a height of 1 m outside the buildings as a function of Aeff



Fig. 3. Exposition gamma dose rates at a height of 1 m inside buildings as a function of Aeff

One needs to remember that the equations (18) and (19) can be used only for calculation of exposition dose rates outside and inside buildings in settlements environment in autumn 1990.

They can be transformed to such equations that will allow the assessment of the exposition dose rates at any arbitrary time *t*.

Let us rewrite the equation (16) in the following form:

$$Aeff(t) = \frac{A^{0}(^{137}Cs)}{K_{\gamma}(^{137}Cs)} \cdot F(t), \qquad (20)$$

where:

 $\begin{array}{ll} A^{\circ} \left( {}^{^{137}}Cs \right) & = \text{initial surface contamination by the} \\ & \text{nuclide} {}^{^{137}}Cs, \\ F(t) & = \text{some function of time.} \end{array}$ 

This function is given by:

$$F(t) = \sum_{i=1}^{n} K_{i,\gamma} \cdot \sigma_i^{\circ} \cdot \exp(-\frac{0.693}{T_{\gamma_2}^i} \cdot t).$$
 (21)

On the basis of the equation (20) one can estimate F(t) as:

$$F(t) = \frac{K \gamma ({}^{137}C s)}{A^{\circ} ({}^{137}C s)} \cdot A eff(t).$$
 (22)

Then the equation (13) can be rewritten as:

$$R_{\gamma}^{\star}(t) = K_{S} \cdot K_{L}(t) \cdot \frac{K_{\gamma}(^{137}Cs)}{A^{\circ}(^{137}Cs)} \cdot Aeff(t).$$
 (23)

The value of  $R_{\gamma}^{*}(t)$  in 1990  $R_{\gamma}^{*}(90)$ , can be calculated from the following formula:

$$R_{\gamma}^{*}(90) = K_{S} \cdot K_{L}(90) \cdot \frac{K_{\gamma}(^{13}CS)}{A^{\circ}(^{13}CS)} \cdot Aeff(90),$$
(24)

where:

 $K_{L}$  (90) = numeric value of shielding factor originated from natural vertical migration of nuclides into the soil,

Aeff(90) = effective contamination of the soil

### in 1990.

On the other hand the equation (18) can be written as:

$$R_{\gamma} (90)_{\text{out}} = R_{\gamma} \text{ bgr.} + R_{\gamma} (90) \text{ch}, \qquad (25)$$

where:

	= exposition dose rate outside the
$R_{\gamma}$ (90) <sub>out</sub>	building in settlements environment
, 040	in autumn 1990,
	= exposition dose rate of background
Rγ <sub>bgr</sub> .	radiation; does not depend on time,
R <sub>v</sub> (90)ch	= contribution of Chernobyl nuclides
1	to the exposition dose rate.

Comparison of the equations (18) and (25) gives:

$$R_{\gamma}(90)_{ch} = 2.381 \cdot Aeff(90)$$
<sup>(26)</sup>

Let us transform the last equation by dividing its left and right parts by  $A^{\circ}({}^{137}Cs)$ :

$$\frac{R_{\gamma}(90)_{ch}}{A^{\circ}(^{137}Cs)} = 2.381 \frac{Aeff(90)}{A^{\circ}(^{137}Cs)}.$$
(27)

The left part of this the equation can be denoted as

 $R_{\gamma}^{*}$  (90)*ch*. It is the exposition dose rate of external radiation normalized to the initial surface contamination by <sup>137</sup>Cs. It is clear that:

$$R_{\gamma}^{*}(90)ch = R_{\gamma}^{*}(90)ch$$
 (28)

Combining the equations (24), (27) and (28) gives the following equation:

$$K_{s} \cdot K_{L}(90) \cdot \frac{K_{\gamma}({}^{137}Cs)}{A^{0}({}^{137}Cs)} \cdot Aeff(90)$$
  
= 2.381 \cdot \frac{Aeff(90)}{A^{0}({}^{137}Cs)}.  
(29)

From here:

.

$$K_s \cdot K_{L}(90) \cdot K_{\gamma}(^{137}Cs) = 2.381.$$
 (30)

The equation (30) can be rewritten as follows:

$$K_{s} = \frac{2.381}{K_{L}(90) \cdot K_{\gamma}(^{137}Cs)}.$$
 (31)

Let us insert the correcting factor  $K_{S}$  determined by the equation (31) into the equation (23). After some changes this gives the following:

$$R_{\gamma}^{*}(t) = \frac{2.381}{K_{L}(90)} \cdot \frac{1}{A^{\circ}(^{137}Cs)} \cdot Aeff^{*}(t), \qquad (32)$$

where  $Aeff^{*}(t)$  is determined by:

$$Aeff^{*}(t) = K_{L}(t) \cdot Aeff(t).$$
<sup>(33)</sup>

Multiplication of the left and right parts of the equation (32) by  $A^{\circ}({}^{137}C s)$  gives:

$$\hat{R}(t)_{ch} = \frac{2.381}{K_{L}(90)} \cdot Aeff^{*}(t), \qquad (34)$$

where:

The numeric value of  $K_{\perp}$  (90) can be estimated by means of the equation (12). It gives  $K_{\perp} = 0.604$ . Insertion of this value into the equation (34) transforms it into:

$$R(t)_{ch} = 3.944 \cdot Aeff^{*}(t), \qquad (35)$$

where:

.

The total exposition dose rate outside the buildings expressed in  $\mu$ R/h can be described by the following equation:

$$R(t)_{\text{out}} = 8.555 + 3.944 \cdot Aeff^{*}(t).$$
(36)

The same procedure allows to develop the equation for calculation exposition dose rates in air inside the buildings (expressed in  $\mu$ R/h):

$$R(t)_{\text{ins}} = 11.814 + 0.836 \cdot Aeff^{*}(t)$$
 (37)

Here the numeric constant 11.814 in the part of the equation (37) determines the average exposition dose rate inside buildings, originated from the background radiation. The second member of the right part of this equation gives the contribution of the Chernobyl nuclides to the total exposition dose rate

inside buildings,  $R(t)_{ch}$  (expressed in  $\mu R/h$ ):

$$R(t)_{ch} = 0.836 \cdot Aeff^{*}(t).$$
 (38)

There are some arguments allowing to state that the equations (35)-(38) can be used for the assessment of the collective doses not only in settlements of the Bryansk oblast but outside of this area. These arguments are discussed below.

Data on measurements of exposition dose rates in air outside and inside were measured by specialists of the "Typhoon" in settlements of the Bryansk oblast that borders the Gomel oblast. That allows the supposition that (35)-(38) are also valid for contaminated territories in the Gomel oblast.

One also needs to consider that the major part of the total deposition of radioactive species in the



Fig. 4 Shielding factors as a function of contamination levels

former USSR lies in the Belarussian-Ukrainian Woodland with quite uniform geochemical characteristics. Moreover, urban and rural settlements in the affected areas of Belarus, Russia and the Ukraine are very similar as well as are the life-style and occupational activities of their inhabitants.

Our assumption on the similarity of irradiation of the Bryansk oblast inhabitants and inhabitants of other affected areas of the former USSR is also supported by the data on the natural radiation background in Belarus prior to the Chernobyl accident [25].

So according to the publication [25], the average exposure dose rate in air in 1981-1982 was 10.6  $\mu$ R/h in urban areas of Minsk, Borisov, Vitebsk, Soligorsk and 9  $\mu$ R/h in rural areas of Minsk rayon. These values very well agree with the dose rate of background radiation (~ 8.6  $\mu$ R/h) that we have estimated for the settlements environment in the Bryansk oblast.

An additional evidence of justified application of the equations (18), (19) and (35)-(38) for assessment of exposition dose rates and doses in the settlements environment outside of the Bryansk oblast gives also a very good agreement between the shielding factors of buildings SF, calculated on the basis of the equations (18), (19) or (36), (37) and estimated on the basis of experimental data.

The shielding factors of buildings are expressed as a ratio of the exposition dose rates inside and outside the buildings.

They can be calculated on the basis of the equations (18), (19) or (35)-(38).

Figure 4 shows the shielding factor estimated by the following equation:

$$SF = \frac{11.814 + 0.836 \cdot Aeff^{*}}{8.555 + 3.944 \cdot Aeff^{*}}.$$
 (39)

As can be seen from Fig. 4 this factor decreases

with the increase of the  $A eff^*$ . Its value changes from about 1.4 at  $A eff^* = 0$  (noncontaminated territory) to 0.4-0.3 in the interval of  $A eff^*$  from 20 to 80 Ci/km<sup>2</sup>.

Table 9 demonstrates the data on the exposition dose rates in air outside of buildings that were calculated for different periods of time on the basis of the equation (35). Here are shown separately individual contributions of short-lived nuclides, isotopes of caesium as well as the total exposition dose rates. Data of Tables 1 and 3 have been also used by calculation of data given in Table 9.

On the basis of Table 9 we have calculated the air-absorbed doses inferred only by  $^{137}Cs$  in settlements environment accumulated over different periods of time  $\tau$  after the Chernobyl accident. Results of this calculation are presented in Table 10. There are also our estimates on the air-absorbed doses in areas of "caesium spots" with the undisturbed soil. They were calculated on the basis of data presented in Table 1 and 3 of our report.

We have also included in Table 10 the data estimated on the basis of data of Miller et al [27] and data used by J. Gofman [12] in his assessment of the Chernobyl consequences.

As it is known, J. Gofman [12] did not carry out a study for estimation of the air-absorbed doses inferred by the isotope <sup>137</sup>Cs. He had based his assessment on investigations conducted by H. Beck and G. de Planque [28] and recommendations of UNSCEAR [29].

Data in Table 10 shows that there is very good agreement between our values of air-absorbed doses estimated for areas with undisturbed soil and values of air-absorbed doses delivered in forest that we estimated on the basis of data [27]. Our doses calculated for areas with undisturbed soil as well very

good agree with data estimated on the basis of data used by J. Gofman [12]. On the other hand, our results for settlements environment fall into intervals formed by data of Miller et al [27] estimated for plowed lands with depth plowing of 15 and 30 cm respectively.

It is also necessary to notice that our air-absorbed doses estimated for settlements environment practically coincide with the arithmetic mean values calculated on the basis of data [27] estimated for different depth of plowing.

For example, air-absorbed doses that we have estimated by use of data of Miller et al [27] for the period of time  $\tau$ =70 years are 0.559 rad for the plowed land with depth plowing of 15 cm and 0.348 rad for depth plowing of 30 cm (see Table 10).

The arithmetic mean of these values is 0.453 rad. The last value is only about 7 percent higher than the value of air-absorbed dose that we have calculated for the settlements environment (0.425).

This agreement is surprising, especially, if one remembers that our estimations are based on experimental data of [24] established in areas of the Bryansk oblast affected by the Chernobyl accident and contaminated by <sup>137</sup>Cs to very high levels.

On the contrary, the data of K. Miller et al [27] have been established in course of experimental investigations of <sup>137</sup>Cs soil-penetration due to global fallout from atmospheric nuclear weapons tests. Moreover, these investigations have been carried out in the USA, which means in areas many thousand

kilometers away from the contaminated territories of the Bryansk oblast.

Taking into consideration all arguments given above one can conclude that our method of dose rates estimation and doses described in this part of our report gives quite correct data.

One can also agree with our statement that the assessment of the collective doses delivered to the population affected by the Chernobyl accident has to be based on results estimated for the settlements environment. The use of data established for undisturbed soil for this purpose will cause an overestimation of the collective dose at least by factor 3. Taking in account this conclusion we shall carry out our further calculation on the basis of data given in Table 9.

The value of the mean equivalent dose of external radiation normalized to the level of contamination by <sup>137</sup>Cs equal to 1 Ci/km<sup>2</sup> can be estimated by use of data from Table 9 on the basis of the equation:

$$H_{\tau \text{ pxt}}^{*} = C_{1} \cdot C_{2} \cdot F_{1} (F_{2} + (1 - F_{2})F_{3}) \cdot P_{\tau} , \quad (40)$$

where:

- $C_1$  = conversion factor from exposition dose to tissue-absorbed dose (rad/R),
- $C_2$  = conversion factor from tissue-absorbed dose to equivalent dose (rem/rad),
- $F_1$  = correction factor (considers different factor that can influence the exposition dose in air, for example snow cover, etc.),

Periods of time, years	Short lived isotopes	<sup>134</sup> Cs	<sup>137</sup> Cs	All nuclides together
50	45.4	98.6	449	593
70	45.4	98.6	486.6	630.6
90	45.4	98.6	508.6	652.6

Table 9. Exposition doses in air at a height of 1 m in "caesium spots", accumulated during different periods of time after the Chernobyl accident (normalized to 1 Ci/km<sup>2</sup> of <sup>137</sup>Cs), in mR.

Table 10.	Calculated	doses of c	aesium-137	in air at a	height of	1 m	accumulated	during	different
r	eriods of tim	e after der	oosition (give	en in rads)	normalize	d to 1	1 Ci/km <sup>2</sup>		

Diago	Pe	eriod of tir	Source of information			
Flace	50	70	90	Infinite time	Source of information	
Forest	1.195	1.365	1.495	_	Kevin M. Miller et all [27]	
Undisturbed field	0.707	0.807	0.866	_	Kevin M. Miller et all [27]	
Plowed land (to 15 cm depth)	0.481	0.559	0.625	_	Kevin M. Miller et all [27]	
Plowed land (to 30 cm depth)	0.296	0.348	0.370		Kevin M. Miller et all [27]	
Undisturbed field	1.058	1.145	1.193		M.Malko, this paper	
Settlement, outside the buildings	0.392	0.425	0.444		M.Malko, this paper	
Used for assessment of the Chernobyl consequences	1.15*	1.25**		1.44	J.Gofman [12]	

Notice: \* this value is evaluated by multiplication of the value 1.44 rad per  $\text{km}^2$  by factor 0.80;

<sup>\*\*</sup> this value is evaluated by multiplication of the value 1.15 rad per  $km^2$  by factor 0.425/0.392.

- $F_2$  = occupancy factor (fraction of time spent outside the buildings, dimensionless),
- $F_3$  = shielding factor of buildings (dimensionless).

 $P_{\tau}$  = exposition dose in air (R).

By carrying out the calculations of  $H_{\tau \text{ pxt}}^{\star}$  we used the following numeric values of these factors: C<sub>1</sub>=0.93 rad/R; C<sub>2</sub>=0.72 rem/rad; F<sub>1</sub>=1; F<sub>2</sub>=0.2 [20], F<sub>3</sub>=0.211.

We estimated the value of factor  $F_3$  by use of the equations (35) and (38). Therefore:  $F_3$  is determined by the equation:

$$F_3 = \frac{0.836 \cdot Aeff^*}{3.944 \cdot Aeff^*} = \frac{0.836}{3.944} = 0,212$$
(41)

As can be seen from the equation (41), the shielding factor of buildings  $F_3$  does not depend on the effective

contamination level  $A \in ff^*$ . It means that F3 is an uniform factor for all contaminated areas. The differences in shielding factors of buildings (39) and (41) arises from the fact, that the equation (39) determines the shielding effect of buildings from combined action of background radiation and Chernobyl nuclides. On the contrary, the equation (41) describes only the shielding effect against radiation caused by nuclides deposited on the ground as a result of the Chernobyl accident. As the task of our study is the assessment of the consequences of the Chernobyl accident we need to separate the influence of background radiation and radiation caused by the accident.

Insertion of the numeric values of the factors  $C_1$ ,  $C_2$ ,  $F_1$ ,  $F_2$  and  $F_3$  into the equation (40) results in the following formula:

$$H_{\tau,\text{ext}}^* = 0.247 \cdot P_{\tau}.$$
 (42)

The next step is estimation of the total mean equivalent dose  $H_{\tau \text{ ext}}^*$  normalized to the initial surface contamination by <sup>137</sup>Cs equal to 1 Ci/km<sup>2</sup>. On analogy to the equation (3) one can write:

$$\overline{H_{\tau}^{*}} = \frac{H_{\tau,\text{ext}}^{*}}{0.6}.$$
(43)

Combination of the two last equations gives:

$$H_{\tau}^{*} = 0.412 \cdot P_{\tau}.$$
 (44)

Application of the equation (44) to a 70 years period of time yields the following value of the normalized mean equivalent dose:  $\overline{H_{\tau}^{*}} = 0.26$  rem / 1 Ci/km<sup>2</sup>. The last value was calculated by use of the value  $P_{\tau} = 0.631 \cdot R$  given in Table 9.

### SOURCE TERM

The total release of nuclides to the environment (source term) is an important parameter in any radiological accident because it determines the scale of this accident. Soon after the Chernobyl accident the Soviet specialists had suggested their data on the amounts of radionuclides that escaped from the destroyed Chernobyl reactor [19]. In this paper these data are presented in Table 11.

According to the estimations of the Soviet specialists about 100 MCi of different nuclides including 50 MCi of radioactive inert gases came into the environment over the 10 days after the first explosion at the reactor of the fourth unit of the Chernobyl NPP.

Later it was recognized that the data published in [19] were underestimated. For example, the UNSCEAR had shown in its report published in 1988 [20], that the total release of <sup>137</sup>Cs into the environment as a result of the Chernobyl accident had been in reality 2 times higher than claimed by the Soviet specialists.

According to the Ukrainian specialists [7] the total release of all radioactive isotopes during the accident at the Chernobyl NPP reached 10,813 PBq (about 296 MCi). This value is about factor 3 higher than the estimations of the specialists of the former USSR in 1986. The data of the Ukrainian specialists on the total release of nuclides from the Chernobyl reactor are presented in this paper in Table 12.

## CAESIUM DEPOSITION IN BELARUS, RUSSIA AND THE UKRAINE.

Over the years that passed since the Chernobyl accident extensive measurements of contamination levels by <sup>137</sup>Cs have been undertaken in Belarus, Russia and the Ukraine. Results of these measurements allowed to establish the mean contamination levels by <sup>137</sup>Cs in many thousand settlements in these countries.

The total area of contaminated territories with initial levels of contamination by <sup>137</sup>Cs that change from some minimal value  $A_{s,\min}^0$  to some maximal

value  $A_{s,\max}^0$  has also been determined.

As a rule 6 different intervals for characterizing of areas of contamination were established:

0.27-0.54 Ci/km<sup>2</sup> (10-20 kBq/m<sup>2</sup>); 0.54-1.0 Ci/km<sup>2</sup> (20-37 kBq/m<sup>2</sup>); 1-5 Ci/km<sup>2</sup> (37-185 kBq/m<sup>2</sup>); 5-15 Ci/km<sup>2</sup> (185-555 kBq/m<sup>2</sup>); 15-40 Ci/km<sup>2</sup> (555-1480 kBq/m<sup>2</sup>); and higher than 40 Ci/km<sup>2</sup> (> 1480 kBq/m<sup>2</sup>).

Nuclide***	Activity of	discharge, MCi	Fraction of activity discharge
	25.04.86	06.05.86**	from reactor on 6 May 1986, %
<sup>133</sup> Xe	5	45	Possibly up to 100
<sup>85m</sup> Kr	0.15		_ ** _
<sup>85</sup> Kr		0.5	_ ** _
<sup>131</sup> I	4.5	7.3	20
<sup>132</sup> Te	4	1.3	15
<sup>134</sup> Cs	0.15	0.5	10
<sup>137</sup> Cs	0.3	1	13
<sup>99</sup> Mo	0.45	3	2.3
<sup>95</sup> Zr	0.45	3.8	3.2
$^{103}$ Ru	0.6	3.2	2.9
<sup>106</sup> Ru	0.2	1.6	2.9
<sup>140</sup> Ba	0.5	4.3	5.6
<sup>141</sup> Ce	0.4	2.8	2.3
<sup>144</sup> Ce	0.45	2.4	2.8
<sup>89</sup> Sr	0.25	2.2	4.0
<sup>90</sup> Sr	0.015	0.22	4.0
<sup>239</sup> Np	2.7	1.2	3.2
<sup>238</sup> Pu	0.1•10 <sup>-3</sup>	0.8•10 <sup>-3</sup>	3%
<sup>239</sup> Pu	$0.1 \bullet 10^{-3}$	$0.7 \bullet 10^{-3}$	- ** -
<sup>240</sup> Pu	0.2•10 <sup>-3</sup>	1●10 <sup>-3</sup>	- " -
<sup>241</sup> Pu	0.02	0.14	_ '' _
<sup>242</sup> Pu	0.3•10 <sup>-6</sup>	<b>2</b> ●10 <sup>-6</sup>	- " -
<sup>242</sup> Cm	3•10 <sup>-3</sup>	$2.1 \bullet 10^{-2}$	- " -

Table 11. Radionuclide composition of discharge from damaged unit of Chernobyl nuclear power plant\* [19]

\*) Error of estimate  $\pm 50\%$ 

\*\*) Total discharge up to 6 May 1986

\*\*\*) The data presented relate to the activity of the main radionuclides measured on radiometric analyses

Table 12.	Radionuclide composition in the active core before the accident and in the total discharge	e
re	calculated on 26.04.1986 [7].	

Inventory of the active core before the accident			Total discharge recalculated on (26.04.1986)		
	(26.04.1986)				
Nuclide	Half life	Activity	Released fraction of	Activity (PBq)	
		(PBq)	inventory, %		
<sup>133</sup> Xe	5.3 d	6 500	100	6500	
<sup>131</sup>	8.0 d	2 300	50-60	-1760	
<sup>134</sup> Cs	2.0 a	180	20-40	-54	
<sup>137</sup> Cs	30.0 a	280	20-40	-85	
<sup>132</sup> Te	78.0 h	2 700	25-60	-1150	
<sup>89</sup> Sr	52.0 d	2 300	4-6	-115	
<sup>90</sup> Sr	28.0 a	200	4-6	-10	
<sup>140</sup> Ba	12.8 d	4 800	4-6	-240	
<sup>95</sup> Zr	1.4 h	5 600	3.5	196	
<sup>99</sup> Mo	67.0 h	4 800	> 3.5	> 168	
<sup>103</sup> Ru	39.6 d	4 800	> 3.5	> 168	
<sup>106</sup> Ru	1.0 a	2 100	> 3.5	> 73	
<sup>141</sup> Ce	33.0 d	5 600	3.5	196	
<sup>144</sup> Ce	285.0 d	3 300	3.5	-116	
<sup>239</sup> Np	2.4 d	27 000	3.5	-95	
<sup>238</sup> Pu	86.0 a	1	3.5	0.035	
<sup>239</sup> Pu	24 400.0 a	0.85	3.5	0.03	
<sup>240</sup> Pu	6 580.0 a	1.2	3.5	0.042	
<sup>241</sup> Pu	13.2 a	170	3.5	-6	
<sup>242</sup> Cm	163.0 d	26	3.5	-0.9	
Total		73559.05		-10933.007	

Notice: The discharge activity recalculated on 6 May 1986 is about 2000 PBq

The data on the total area of contaminated territories of Belarus, Russia and the Ukraine characterized by these intervals of contamination levels by <sup>137</sup>Cs are presented in Tables 13-16.

The data given there allow to estimate the minimal amounts of <sup>137</sup>Cs deposited in Belarus, Russia and the Ukraine by means of the following equation:

 $Q_{\min}^{i} = A_{s,\min}^{0,i} \cdot S_{i}, \qquad (45)$ 

where:  $O_{m}^{i}$ 

$$i$$
 = the minimal amounts of <sup>137</sup>Cs deposited  
on the area with the initial contamination  
by <sup>137</sup>Cs acqual to  $2^{0}i$  (Ci):

$$S_{i}$$
 = the area of territories contaminated by  
<sup>137</sup>Cs at the level  $A_{Smin}^{0,i}$  (km<sup>2</sup>).

The results of our calculation on the basis of the equation (45) are given in Table 17. In case of Belarus this method gives  $Q_{\min}^i = 260,000 \text{ Ci}$ .

Division of this minimal deposition value of  ${}^{137}$ Cs,  $Q_{m in}^{i}$  by the total area of Belarus (207.6 thousand

square kilometers) gives the following minimal average contamination of the total territory of Belarus by  $^{137}Cs$ :

$$A_{s,\min} \approx 1.25 \text{ Ci} / \text{km}^2 \text{ or } 46.3 \text{ kBq} / \text{m}^2$$

It is interesting to notice that the UNSCEAR performed its assessment of the collective irradiation doses originated from the Chernobyl accident by the average contamination of Belarus equal to 39 kBq/m<sup>2</sup> [20]. This value is about 20 percent lower than the value 46.3 kBq/m<sup>2</sup> which was calculated on the basis of the equation (45).

It is clear that the real average contamination of Belarus by  $^{137}$ Cs has to be higher than 1.25 Ci/km<sup>2</sup>. We have developed a simple method for estimation of more correct data on the total contamination of Belarus by  $^{137}$ Cs.

The corrected amount of the deposited  $^{137}$ Cs, Q<sub>tot</sub>, as a result of the Chernobyl accident can be calculated by the following equation:

$$Q_{0} = \sum_{i=1}^{N} \overline{A_{s,i}^{0}} \cdot \Delta S_{i}$$
(46)

 Table 13. Contaminated areas in Belarus with the level of caesium-137 equal to 1 Ci/km<sup>2</sup> or higher (square kilometers) [31].

Oblast		Level of contamination, Ci/km <sup>2</sup>			
Oblast	1-5	5-15	15-40	> 40	
Brest	3800	470			
Vitebsk	35				
Gomel	16870	6740	2760	1625	
Grodno	1690	12			
Minsk	2030	48			
Mogilev	5490	2900	1450	525	
Total	29915	10170	4210	2150	

Table 14. Contaminated areas in the Russian Federation with the level of caesium-137 equal to 1 Ci/km<sup>2</sup> or higher (square kilometers) [31].

Oblast		Level of cont	tamination, Ci/km <sup>2</sup>	
Oblast	1-5	5-15	15-40	> 40
Belgorod	1620			
Bryansk	6750	2628	2130	310
Voronezh	1320			
Kaluga	3500	1419		
Kursk	1220			
Lipetsk	1690			
Leningrad	850			
Nizhni Novgorod	15			
Orel	8840	132		
Penza	4130			
Ryazan	5210			
Saratov	150			
Smolensk	100			
Tambov	510			
Tula	10320	1271		
Uljanovsk	1060			
Total	48915	5450	2130	310

where:

*N* = number of all contaminated settlements;

- $\overline{A_{s,i}^{0}}$  = the average level of initial surface contamination by <sup>137</sup>Cs in *i*th settlement, (Ci/km<sup>2</sup>);
- $\Delta S_{i}$  = area of the territory that belonged to *i*th settlement (km<sup>2</sup>).

The equation (46) can be simplified in order to make the estimation of the value Q easier.

It is known that there is no significant difference in the population density in the affected regions of Belarus. The same conclusion can be made in relation to Russia as the Ukraine. It is also known that the majority of settlements in the affected regions have quite similar number of inhabitants.

Considering the abovementioned facts we can estimate the value of  $\Delta S_i$  by:

$$\Delta S_{i} \approx \frac{S}{N} , \qquad (47)$$

where:

S = the total area of all contaminated territories. Combination of the equations (46) and (47) gives:

$$Q_{0} \approx S \cdot \sum_{i=1}^{N} \frac{\overline{A_{S,i}^{0}}}{N} = S \cdot \overline{A_{S}^{0}}, \qquad (48)$$

where:

 $\overline{A_s^0}$  = contamination level by <sup>137</sup>Cs averaged over the total contaminated area (Ci/km<sup>2</sup>). As can be seen from the equation (48) the value of

 $A_{s}^{0}$  is determined by the equation:

$$\overline{A_{S}^{0}} = \sum_{i=1}^{N} \frac{\overline{A_{S,i}^{0}}}{N} .$$
 (49)

The last equation was used for the calculation of the averaged levels for areas with the level of contamination  $A_s^0 \ge 1 \text{Ci/km}^2$ . Results of the calculation performed on the basis of the experimental data on  $\overline{A_{s,i}^0}$  presented in the publication [32] are given in Table 18. There are also the averaged levels of contamination by <sup>137</sup>Cs at levels of contamination  $A_s^0 \le 1 \text{Ci/km}^2$ . In this case the following equation was used:

Table 15. Contaminated areas in the Ukraine with the level of caesium-137 contamination equal to 1Ci/km² (square kilometers) [31].

Oblast		Level of contam	nination, Ci/km <sup>2</sup>	
	1-5	5-15	15-40	> 40
Vinnitsy	1944	38		
Volyn	582			
Dnepropetrovsk	38			
Donetsk	410			
Zhitomir	9192	1780	336	154
Ivano-Frankovsk	606			
Kiev	7695	957	546	417
Kirovograd	219			
Nikolaev	24			
Odessa	27			
Rovno	9332	181		
Sumy	491			
Ternopol	357			
Cherkasy	3233	72		
Chernigov	2221	135		
Chernovtsy	500	14		
Kharkov	31.4	16		
Chmelnitsk	319			
Total	37205	3177	882	571

Table 16. Caesium-137 contaminated areas in Belarus, the Russian federation and the Ukraine with level 10-37 kBq/m<sup>2</sup>[6].

Countries	Area (in 1000 km <sup>2</sup> ) contaminated above specified levels			
	10-20	20-37		
Belarus	60	30		
Russia	300	100		
Ukraine	150	65		

Range of contamination,		Q, Ci	Q, Ci		
Ci/km <sup>2</sup>	Belarus	Russia	Ukraine		
0.27-0.54 (10-20 kBq/m <sup>2</sup> )	16200	81000	40500		
0.54-1 (20-37 kBq/m <sup>2</sup> )	16200	54000	35100		
1-5	29915	48915	37205		
5-15	50850	27250	15885		
15-40	63150	31950	13230		
> 40	86000	12400	22840		
Total deposition	262315	255515	164760		
Rounded total deposition, Ci	260000	255000	165000		

 Table 17. Minimal deposition of caesium-137 on territories of Belarus, the Russia Federation and the Ukraine

Range of contamination levels, Ci/km <sup>2</sup>	Average contamination level, Ci/km <sup>2</sup>
0.27-0.54 (10-20 kBq/m <sup>2</sup> )	0.405
0.54-1.0 (20-37 kBq/m <sup>2</sup> )	0.77
1-5	2.98
5-15	8.58
15-40	24.30
> 40	52.7

 Table 19.
 Total deposition of caesium-137 on territories of Belarus, the Russian Federation and the Ukraine.

Range of contamination		Q, Ci	
levels, Ci/km <sup>2</sup>	Belarus	Russia	Ukraine
0.27-0.54 (10-20 kBq/m <sup>2</sup> )	24300	121622	60810
0.54-1 (20-37 kBq/m <sup>2</sup> )	23100	77027	50066
1-5	89147	145767	110871
5-15	87259	46761	27259
15-40	102305	51759	21433
> 40	113305	16337	30092
Total deposition, Ci	439414	459273	300531
Rounded total deposition, Ci	440000	460000	300000

$$\overline{A_{s}^{0}} = \frac{A_{s,\min}^{0} + A_{s,\max}^{0}}{2}.$$
 (50)

By use of the data given in Tables 13-16 and 18 data on the initial contamination of the affected areas in Belarus, Russia and the Ukraine were calculated. Table 19 shows the results of this calculation. The data given in Tables 17 and 19 show that the described method of calculation has given increased amounts of deposited 137Cs in Belarus, Russia and the Ukraine by factor 1.7 (Belarus) -1.8 (Russia, Ukraine) estimations made on the basis of the equation (45).

One needs to notice that the amount of <sup>137</sup>Cs deposited in the Ukraine which is shown in Table 19 does not include big amounts of this isotope in the 30 kilometer zone. According to the Ukrainian National Report [7] there is about 470 thousand Ci of <sup>137</sup>Cs in temporary storages of radioactive materials, in the soil, in hydrosystems, etc.

By taking into account this value and the data on the deposition of <sup>137</sup>Cs in Belarus, Russia and the Ukraine

given in Table 19 as well as the data on the deposition of this isotope outside of the former USSR (1.2 MCi [20]) one can assess the approximate amounts of <sup>137</sup>Cs that was discharged from the destroyed Chernobyl reactor. It reaches 2.87 MCi. Summing up this number with the amounts of <sup>137</sup>Cs deposited in the Baltic States and Countries of the CIS other than Russia, Belarus and the Ukraine one can assess the total discharge of <sup>137</sup>Cs to the environment as a result of the Chernobyl accident equal to about 3 MCi of the core inventory of this nuclide before the accident. The last figure coincides with upper limit of the estimations given in publication [7].

### **COLLECTIVE DOSE ASSESSMENT**

As it was shown earlier, the collective irradiation dose can be calculated by means of the equation (1). It is possible to use for this purpose the equation:

$$H_{\tau}^{\text{coll}} = N \cdot H_{\tau}^{*} \cdot A_{S}^{0}$$
(51)

Insertion of the value  $A_s$  given by (6) into the equation (51) gives the following:

$$H_{\tau}^{\text{coll}} = N \cdot \overline{H_{\tau}^{*}} \cdot \frac{Q^{0}}{S}$$
(52)

The total number of the inhabitants of the affected areas N can be estimated by the following:

$$N = \rho \cdot S , \qquad (53)$$

where:

 $\overline{\rho}$  = the average population density in contaminated areas (man/km<sup>2</sup>).

The combination of equations (52) and (53) gives:

$$H_{\tau}^{\infty \mathbb{I}} = \rho \cdot H_{\tau}^{*} \cdot Q^{0}$$
(54)

Results of the collective equivalent doses calculation that can be delivered to affected population of Belarus, Russia and the Ukraine within the period of 70 years after the Chernobyl accident are obtained on the basis of the equation (54) and are given in Table 20. This table also contains data on the total deposition of the isotope <sup>137</sup>Cs in Belarus, Russia and the Ukraine, the total areas of contaminated territories of these states as well as the data on the mean contamination levels, mean population density and mean dose delivered over 70 years on a territory with the contamination level by <sup>137</sup>Cs equal to 1 Ci/km<sup>2</sup>.

The data on population density presented in Table 20 were calculated for 1986 on the basis of data of the statistical handbook of the USSR [33]. Analysis of data given in Table 20 shows that the highest equivalent irradiation dose has to be delivered to the Ukrainian population. It can be estimated as  $6.6 \cdot 10^4$  man·Sv over 70 years. The collective equivalent irradiation doses of inhabitants of the affected areas of Belarus and Russia according to our estimation are  $5.5 \cdot 10^4$  and  $4.4 \cdot 10^4$  man·Sv respectively. At the same time the highest mean individual equivalent dose can be delivered to inhabitants of the contaminated territories of Belarus. It is practically 3 times higher than the respective values estimated for

Russia and the Ukraine.

# ASSESSMENT OF COLLECTIVE THYROID DOSES

The assessment of collective thyroid doses delivered to the population of Belarus, Russia and the Ukraine can be performed on the basis of the data established in the course of extensive studies by V. Stepanenko, A. Tsyb, Yu. Gavrilin et al. [34].

By use of the experimental data on [31] accumulation in thyroid measured within the first days after the nuclear explosion at the Chernobyl accident these Russian specialists estimated the collective thyroid dose of 3,674,000 residents of contaminated territories of 7 oblasts of Russia (Bryansk, Tula, Kaluga, Orel, Riasan, Kursk, Leningrad) as 234,000 person·Gy. [34]. This value can be used for the estimation of the factor q<sub>th</sub> that determines the collective thyroid dose of <sup>131</sup>I.

In order to fulfill this task one needs to determine the total deposition of  $^{131}$ I in "7 oblasts" of the Russian Federation. It can be done by means of the equation:

$$Q_{7}^{0}(^{131}I) = \psi \cdot Q_{7}^{0}, \qquad (55)$$

where:

 $Q_{7}^{0}(^{131}\mathcal{I}) = \text{total deposition of }^{131}\text{I in 7 oblasts}$ of Russia,

 $Q_{7}^{\circ}$  = total deposition of <sup>137</sup>Cs in 7 oblasts of Russia,

 $\psi$  = correlation factor averaged over the total area of contaminated territories of "7 oblasts" expressed as a ratio of surface contamination level by <sup>131</sup>I to surface contamination by <sup>137</sup>Cs

Experimental values of the factor  $\psi$  at 23.05.1986 measured in different settlements of Belarus and Russia [35] are given in Table 21 of this report.

Table 22 shows the values of the factor  $\psi$  recalculated to 26.04.1986. On the basis of these values we have estimated the mean values of factor  $\psi$ 

Parameter **Belarus** Russia Ukraine Total deposition of caesium-137, Ci 440000 460000 300000 Total area of contaminated territories with level of caesium-137  $\ge$  0.27 Ci/km<sup>2</sup> (10 KBq/m<sup>2</sup>), in k m<sup>2</sup> 136445 348915 256835 Mean contamination level, Ci/km<sup>2</sup> 3.225 1.168 1.318 Mean dose equivalent commitment during 70 years 0.26 after the accident (normalized to 1 Ci/ km<sup>2</sup>), rem 0.26 0.26 Mean population density, person/km<sup>2</sup> 84.5 48.2 36.6 12770289 21702558 Total number of people living in contaminated areas 6576649 Collective dose, manrem 5514520 4376123 6590633 Collective dose in man Sv, rounded 5.5•10<sup>4</sup>  $4.4 \bullet 10^4$ 6.6•10<sup>4</sup> Mean individual dose, rem 0.84 0.26 0.30

 Table 20. Collective equivalent irradiation doses of populations of Belarus, the Russian Federation and the Ukraine.

Defail up un	u the Rus	siun i cuc	1 <b>u</b> tion by <b>2</b> 5.0	0.1700 [00].			
Settlement	<sup>131</sup> I	$^{103}$ Ru	<sup>134</sup> Cs	$^{95}$ Zr + $^{95}$ Nb	$^{140}La + {}^{140}Ba$	<sup>90</sup> Sr	
	Belarus						
Gomel	3.48	1.74	0.42	4.54	1.97	0.17	
Mozyr	1.16	1.0	0.39	1.06	1.06	0.13	
Pinsk	5.68	1.98	0.58	2.28	1.0	_	
Zhitkovichi	1.78	3.89	0.53	2.22	1.67	_	
Khoiniki	1.62	3.49	0.56	11.2	4.6	0.07	
Jurovichi	1.47	2.50	0.43	6.47	2.21	0.06	
		R	ussia, Bryansk (	oblast			
Barsuki	0.64	0.86	0.54	0.13	0.30	0.11	
Makarichi	0.74	0.93	0.4	0.08	0.39	_	
Novozybkov	0.88	0.85	0.46	0.07	0.38	_	
Nikolaevka	0.82	0.83	0.55	0.09	0.36	0.008	
Berezovka	2.05	1.58	0.55	0.14	0.34	0.008	
Svjatsk	1.17	1.11	0.56	0.13	0.31	0.03	
Bartolomeevka	0.68	1.0	0.63	0.14	0.06	0.04	
		R	ussia,Kaluga ob	olast			
Zhisdra	0.91	1.0	0.52	0.03	0.25		
Mileevo	0.95	0.85	0.53	0.1	0.37	_	
Kolodjassy	0.95	1.01	0.56	0.07	0.31	_	
RussiaTula oblast							
Belev	1.03	1.34	0.54	0.06	0.23		
Plavsk	0.55	0.69	0.53	0.06	0.45		
Dubovka	0.63	1.02	0.54	0.14	0.14		
Uzlovaja	1.26	2.55	0.45	0.05	0.26	<u> </u>	

 Table 21.
 Ratios of nuclide activities to Cs-137 activity in soil of different contaminated areas of Belarus and the Russian Federation by 23.05.1986 [35].

 Table 22.
 Calculated values of I-131/Cs-137 activity ratio in contaminated areas of Belarus and the Russian Federation by 26.04.1986.

	<u>I-131</u>	<u>I-131</u>	
Settlement	Cs-137	Cs-137	
	(23.05.86)	(26.04.86)	
	Belarus		
Gomel	3.48	38.9	
Mozyr	1.16	12.96	
Pinsk	5.68	63.45	
Zhitkovichi	1.78	19.9	
Khoiniki	1.62	18.1	
Jurovichi	1.47	16.4	
	Russia, Bryansk oblast	t	
Barsuki	0.64	7.15	
Makarichi	0.74	8.27	
Novozybkov	0.88	9.83	
Nikolaevka	0.82	9.16	
Berezovka	2.05	22.9	
Svjatsk	1.17	13.1	
Bartolomeevka	0.68	7.60	
	Russia, Kaluga oblast		
Zhisdra	0.91	10.2	
Mileevo	0.95	10.6	
Kolodjassy	0.95	10.6	
Russia, Tula oblast			
Belev	1.03	11.7	
Plavsk	0.55	6.14	
Dubovka	0.63	7.04	
Uzlovaja	1.26	14.1	

for Russia and Belarus as 10 and 20 respectively.

By use of factor  $\psi$ =10 and the data given in Tables 14 and 18 we have calculated the total deposition of

<sup>131</sup>I in "7 oblasts" of Russia as 2.24 MCi. By estimation of this value we did not take into consideration the contaminated territories in "7

Parameter	Belarus	Russia	Ukraine
Total deposition, Ci	8800000	4600000	6000000
Total area of contaminated territories, km <sup>2</sup>	136445	456805	256835
Mean population density, person/km <sup>2</sup>	48.2	36.6	84.5
Collective thyroid dose equivalent commitment			
per unit release, man.Sv/Bq	3.90·10 <sup>-12</sup>	$2.95 \cdot 10^{-12}$	$6.80 \cdot 10^{-12}$
Thyroid collective dose, man.Sv	$12.7 \cdot 10^5$	$5.0 \cdot 10^5$	$15.0 \cdot 10^5$
Collective effective dose equivalent resulted			
from thyroid irradiation, man.Sv	$6.35 \cdot 10^4$	$2.5 \cdot 10^4$	$7.5 \cdot 10^4$
Mean individual dose of thyroid irradiation, rem	19.3	3.0	6.9

 Table 23. Collective thyroid dose of the affected populations in Belarus, the Russian Federation and the Ukraine.

Table 24.	Contributions of different nuclides to collective effective equivalent doses of irradiation	n
d	ring the period of 70 years after the Chernobyl accident.	

Nuclide	Belarus	Russia	Ukraine
Short lived isotopes other than <sup>131</sup> I, man.Sv	$0.4 \cdot 10^4$	$0.38 \cdot 10^4$	$0.48 \cdot 10^4$
<sup>134</sup> Cs, man.Sv	$0.86 \cdot 10^4$	$0.68 \cdot 10^4$	$1.03 \cdot 10^4$
<sup>137</sup> Cs, man.Sv	$4.24 \cdot 10^4$	$3.34 \cdot 10^4$	$5.09 \cdot 10^4$
<sup>131</sup> I, man.Sv	$6.35 \cdot 10^4$	$2.5 \cdot 10^4$	$7.5 \cdot 10^4$
Total collective effective equivalent dose,			
man.Sv	$11.85 \cdot 10^4$	$6.9 \cdot 10^4$	$14.1 \cdot 10^4$

\*) Notice: these values were estimated by multiplication of collective thyroid doses by organ weighting factor 0.05 [36].

 Table 25.
 Collective effective dose equivalent commitments per unit release of caesium and iodine nuclides.

Country,	Period of time,	<sup>134</sup> Cs	<sup>137</sup> Cs	$^{131}$ I
organization	years	man.Sv		
Belarus	50	$1.06 \cdot 10^{-12}$	$2.42 \cdot 10^{-12}$	$1.95 \cdot 10^{-13}$
	70	$1.06 \cdot 10^{-12}$	$2.61 \cdot 10^{-12}$	$1.95 \cdot 10^{-13}$
Russia	50	$8.04 \cdot 10^{-13}$	$1.82 \cdot 10^{-12}$	$1.47 \cdot 10^{-13}$
	70	$8.04 \cdot 10^{-13}$	$1.96 \cdot 10^{-12}$	$1.47 \cdot 10^{-13}$
Ukraine	50	$1.86 \cdot 10^{-12}$	$4.23 \cdot 10^{-12}$	$3.38 \cdot 10^{-13}$
	70	$1.86 \cdot 10^{-12}$	$4.59 \cdot 10^{-12}$	$3.38 \cdot 10^{-13}$
UNSCEAR, 1988	50	$3 \cdot 10^{-12}$	6·10 <sup>-12</sup>	$1 \cdot 10^{-13}$

oblasts" of Russia with the contamination levels by <sup>137</sup>Cs less than 1 Ci/km<sup>2</sup>.

Division of the collective thyroid dose established by V. Stepanenko, A. Tsyb, Yu. Gavrilin et al [34] for the residents of "7 oblasts" by the assessed deposition of <sup>131</sup>I one can obtain the value of  $q_{th}$  factor equal to  $2.82 \cdot 10^{-12} \cdot Gy / Bq$  or approximately  $2.82 \cdot 10^{-12} \cdot Sv / Bq$ .

In order to emphasize the fact that the numerical values of  $q_{th}$  given above have been estimated on the basis of the data established by [34] for residents of "7 oblasts" we shall describe it as  $q_{th,7}$ .

Multiplication of  $q_{th,7}$  by the thyroid weighting factor 0.05 [36] converts it to the collective effective dose equivalent commitment per unit release of <sup>131</sup>I equal to  $1.41 \cdot 10^{-13}$  man·Sv/Bq. The last value only insignificantly differs from the collective effective dose equivalent commitment per unit release of <sup>131</sup>I estimated in 1988 by UNSCEAR in its assessment of the Chernobyl accident consequences. This surprising agreement is a very important evidence of sound scientific backgrounds of the methods used by UNSCEAR [20] and by V. Stepanenko, A. Tsyb, Yu. Gavrilin et al in estimation of collective thyroid doses resulted from the Chernobyl accident. It also states that the numeric value of  $q_{th,7}$ =  $2.82 \cdot 10^{-12}$  man·Sv/Bq can be used as the reference value of this factor. It is necessary to notice that the factor  $q_{th,7}$  is a function of population density. Therefore, one needs to correct the numeric value of  $q_{th,7}$  in case of population density  $\rho$  that differs from the average population density in "7 oblasts" of the Russian Federation. It can be made by means of the following equation:

$$q_{th} = q_{th,7} \cdot \frac{\rho}{\rho_7}$$
, (56)

where:

- $q_{th}$  = the collective thyroid dose commitment per unit of <sup>131</sup>I deposition (man·Sv/Bq) by arbitrary population density,
- $q_{th,7}$  = the collective thyroid dose commitment per unit of <sup>131</sup>I deposition (man·Sv/Bq) established on the basis of "7 oblasts" data,

- $\rho$  = population density in contaminated area (persons per km<sup>2</sup>),
- $\rho_7 \qquad = \text{population density in contaminated areas of} \\ \text{``7 oblasts'' of the Russian Federation.}$

Table 23 presents the data on the collective thyroid doses estimated for population of Belarus, Russia and the Ukraine by means of factor  $q_{th}$  calculated on the basis of the equation (56).

These data can be possibly considered as upper limits of collective thyroid doses delivered to the populations of Belarus, Russia and Ukraine.

The total amounts of <sup>131</sup>I deposited in these countries were calculated by means of the data on <sup>137</sup>Cs deposition presented in Table 19 as well as the following numeric values of factor  $\psi$ : 10 for Russia and 20 for Belarus and the Ukraine.

As can be concluded from the above-discussed facts, the correctness of the collective thyroid dose estimations is fully determined by the correctness of <sup>131</sup>I depositions. This means that each improvement in the estimation of the total amounts of <sup>131</sup>I deposited in areas affected by the Chernobyl accident will improve estimation of the collective thyroid doses.

The results of our estimations given in Table 23 show that the highest collective thyroid dose was delivered to the Ukrainian population and the lowest to the population of Russia. On the contrary, as can be seen from Table 23, the highest mean individual thyroid dose was estimated for the residents of the contaminated territories of Belarus. It is higher by factor 6.4 than the mean individual thyroid dose in Russia and by factor 2.8 than the mean individual thyroid dose in the Ukraine.

Table 23 as well contains the data on the collective effective dose equivalents resulted from irradiation of people by <sup>131</sup>I in Belarus, Russia and the Ukraine. Table 24 shows the individual contributions of different isotopes to the total collective effective equivalent doses calculated on the basis of data given in the tables 20 and 23. The collective effective dose equivalent commitments per unit release of caesium and iodine nuclides are shown in Table 25.

## LIFETIME MORTALITY RISK FROM SOLID CANCERS AND LEUKEMIA

The latest estimations of the risk coefficient of radiation induced solid cancers and leukemia are shown in the Table 26. In our prognosis of delayed radiological effects we have used the data of the UNSCEAR 94 [37] that are 2-fold less than the data estimated in 1994 by J. Gofman [38].

We preferred the UNSCEAR 1994 risk coefficient of radiation-induced solid cancers due to the following reason. It is well known that the risk coefficient of solid cancers induced by radiation is proportional to the frequency of spontaneous cancers which is a function of the life expectancy. J. Gofman [38] had obtained his value on the basis of the Vital Tables of the USA population. However, the life expectancy of the American population is 10-15 years higher than the life expectancy in Belarus, Russia and the Ukraine [33, 39].

So, even in case the UNSCEAR will in future increase its estimations of the risk coefficient of radiation-induced solid cancers one needs to remember that estimations of the UNSCEAR are based on the data established for the atomic bomb survivors that have been living in a country with a very high life expectancy in comparison to the life expectancy of the people in Belarus, Russia and the Ukraine. One also needs to remember that in contrast to the USA, Japan and other developed countries characterized by a permanent increase of life expectancy, Belarus, Russia and the Ukraine have a permanent decrease of life expectancy.

### PROGNOSIS OF STOCHASTIC EFFECTS

The estimations of the possible stochastic effects in the effected areas of Belarus, Russia and the Ukraine are given in Table 27. The data on morbidity and mortality from radiation-induced thyroid cancers were calculated by use of the data on the collective thyroid doses presented in Table 23 and the risk coefficients  $160 \cdot 10^{-4} \text{Sv}^{-1}$  and  $16 \cdot 10^{-4} \text{Sv}^{-1}$ , respectively. The latter coefficients were estimated by the following method. According to the data given in the Table B-17 of ICRP Publication 60 [36] (see p.132 of Publication 60) the risk coefficient of fatal thyroid cancer for low dose, low dose rate and Low LET radiation (DDREF is equal 2) is  $8 \cdot 10^{-4} \text{Sv}^{-1}$ . For the case of an acute or quasi-acute irradiation (Chernobyl case) one needs to use as a risk coefficient of fatal thyroid cancer a value 2 fold higher or 16.10<sup>-4</sup>Sv<sup>-1</sup> (DDREF=1). From here one can receive the lifetime morbidity risk coefficient 160.10<sup>-4</sup>Sv<sup>-1</sup> as used for calculation of data shown in Table 27.

The data on mortality from solid cancers other than thyroid cancer were used on the basis of the risk coefficient from solid cancers estimated in 1994 by the UNSCEAR and "corrected" for fatal thyroid cancers. The "correction" was in reality a subtraction of the value  $16 \cdot 10^{-4} \text{Sv}^{-1}$  from the value  $1,090 \cdot 10^{-4} \text{Sv}^{-1}$ estimated by the UNSCEAR. One can receive on this simplified way the value  $1,074 \cdot 10^{-4} \text{Sv}^{-1}$  as a risk coefficient for fatal solid cancers other than thyroid cancers. Such "corrected" risk coefficient was used in assessment of fatal cancers in Belarus, Russia and the Ukraine resulted from the Chernobyl accident.

By the calculation of leukaemia mortality we used the value  $110 \cdot 10^{-4}$ Sv<sup>-1</sup> determined by the UNSCEAR in 1994 [37].

According to our estimations the Chernobyl accident will cause about 20,000 additional thyroid cancers among children and adults of the affected areas of Belarus. About 10 per cent of this number can be fatal (2,000 fatal thyroid cancers). We have estimated the number of radiation-induced thyroid cancers in Russia as 8,000 (800 fatal thyroid cancers). For the Ukraine we predicted 24,000 additional thyroid cancers (2,400 fatal thyroid cancers). Considering our method of collective thyroid doses assessment one can believe that these data on the number of additional thyroid cancers are upper limits of possible thyroid cancers.

As can be seen from this Table, the numbers of additional fatal thyroid cancers in each state is similar to the numbers of additional leukaemia and about 10 times less than the numbers of additional fatal solid cancers other than thyroid cancer.

We have assessed the total number of radiation-induced fatal cancers and leukaemia in

Belarus, Russia and the Ukraine, as a result of the Chernobyl accident as about 44,000 with 16,030 cases in Belarus, 8,960 in Russia and 19,050 in the Ukraine.

As it was said above we have carried out our assessment only for populations living in the contaminated areas of these countries. We have estimated the total deposition of  $^{137}$ Cs in these areas as 1.2 MCi (total deposition of <sup>134</sup>Cs about 0.6 MCi). The same amount of <sup>137</sup>Cs deposited in the countries outside the former USSR. If we assume that the number of radiation-induced cancers and leukaemia outside the former USSR caused by the Chernobyl accident will be the same as the total number of solid cancers and leukaemia in Belarus, Russia and the Ukraine because of similar deposition of <sup>137</sup>Cs, we will also obtain about 44,000 additional cancers and leukaemia for the countries outside the former USSR and about 90,000 fatal cancers for all the countries of the world including the former USSR. This assumption means that the Chernobyl accident will

Table 26.Lifetime solid cancers and leukemia mortality risk following acute whole body exposure<br/>to 1 Sv.

Projection method	Lifetime risk, %	Data source	
Solid cancers			
Constant relative risk	10.9	[37]	
Decline to risk for age at exposure 50 years	9.2	[37]	
Decline to zero risk at age 90 years	7.5	[37]	
Constant relative risk (UNSCEAR 1988)	9.7	[20]	
Whole body Cancer Dose (J. Gofman - 1981)	37.31	[12]	
Whole body Cancer Dose (J. Gofman - 1990)	26.64*	[18]	
	25.56**	[18]	
Whole Body Cancer Dose (J. Gofman - 1994)	23.37*	[38]	
	22.35**	[38]	
	Leukemia		
Linear-quadratic dose response model	1.1	[37]	
Constant relative risk (UNSCEAR 1988)	1.0	[20]	

Notice: \*) values determined by using T65DR dosimetric data of the RERF

\*\*) values determined by using DS86 dosimetric data of the RERF

Table 27. Forecast of stochastic effects in Belarus, the Russian Federation, and the Ukraine as a result of the Chernobyl accident (DDREF = 1).

Effect	Belarus	Russia	Ukraine
Thyroid cancer (morbidity)	20300	8000	24000
Thyroid cancer (mortality)	2030	800	2400
Leukemia (mortality)	1300	760	1550
Solid cancers other than thyroid cancer (mortality)	12700	7400	15100
All cancers and leukemia (mortality)	16030	8960	19050

Table 28. Forecast of stochastic effects in Belarus, the Russian Federation, and the Ukraine as a result of the Chernobyl accident (DDREF = 2).

Effect	Belarus	Russia	Ukraine
Thyroid cancer (morbidity)	10150	4000	12000
Thyroid cancer (mortality)	1010	400	1200
Leukemia (mortality)	650	380	775
Solid cancers other than thyroid cancer (mortality)	6350	3700	7550
All cancers and leukemia (mortality)	8010	4480	9525

cause in the affected countries the number of fatal cancers and leukaemia which is similar to the death numbers resulted from atomic bombardment of Hiroshima or Nagasaki.

One may also assume that the total number of potential victims in case of such accident in a country with very high life expectancy and high population density, for example in Japan, will be much higher as in the case of the Chernobyl accident.

The data discussed above have been estimated by use of the factor DDREF=1, which is recommended by the ICRP for the case of acute irradiation by high doses and dose rates [36]. For chronic irradiation of the population by low doses the ICRP recommends the value of the reduction factor DDREFF equal to 2.

In the light of new data [37, 40, 41] it seems that even in cases of low doses and dose rates this factor is very close to 1 for solid cancers. But for leukaemia the DDREF factor is about 2.5 or even higher [37, 41]. For comparison only we have also carried out the estimations of additional stochastic effects assuming that DDREF=2. The results of such estimation are presented in Table 28.

We understand that the results of our assessment have rather qualitative than quantitative character because of many limitations in our study. For example we did not consider the possible interaction of radiation and chemical contamination in the affected areas of Belarus, Russia, and the Ukraine. This can be a reason for the very significant difference in predicted and manifested stochastic effects among the people affected by the Chernobyl accident. However, at present, there is no such quantitative information that could be used in assessment of radiological consequences of this accident in the areas having a very strong chemical contamination. Because of a lack of necessary information we also did not consider deterministic effects that could be even more important as radiological consequences in comparison with stochastic effects as well as genetic affects and effects of a very strong increase in the incidence of general somatic diseases in the affected areas established by Belorussian, Russian and the Ukrainian specialists soon after the accident. At the same time we believe that our data are quite accurate for solving of some problems that arose due to the Chernobyl accident. It is well known, that the reliable data on the increase of the morbidity on thyroid cancer among children of the affected areas of Belarus were established already at the end of the 80's [42,43]. Now, practically all specialists in the field of radiation biology and medicine recognise that this increase is the result of the Chernobyl accident [44]. However, at the end of 80's no such increase in the morbidity on thyroid cancer was established in Russia or in the Ukraine. This fact was considered by specialists of many countries as an evidence of incorrectly assessed data.

The difference in the change of the morbidity on thyroid cancer can be explained very easy on the basis of the data shown in Table 23. As it was said earlier, the highest mean individual thyroid doses were delivered by the Chernobyl accident to the residents of the affected areas of Belarus and the lowest to the Russian population. It is known that the latent periods of stochastic effects induced by the radiation depend from the irradiation doses. The lower the irradiation dose is, the longer is the latent period. This is an explanation why marked increase in the thyroid cancer incidence was registered first in Belarus and then in the Ukraine and later in Russia.

Our data can also answer the question why there are no reliable data on additional leukaemia among the residents of the affected areas of Belarus, Russia and the Ukraine up to present time [45-47]. This contrasts the data established among the Hiroshima and Nagasaki inhabitants that survived the atomic bombardment. Additional cases of leukaemia among them were registered firstly, and only then the radiation-induced solid cancer.

We suggest the following explanation for these differences in the manifestation of stochastic effects among the irradiated populations of Belarus, Russia, the Ukraine and the residents of Hiroshima and Nagasaki.

The survivors of the atomic bombardment of these Japanese cities received practically the same doses on bone marrow (the whole body dose) and on thyroid. The more earlier manifestation of leukaemia than thyroid cancers by inhabitants of Hiroshima and Nagasaki indicates that the latent period of leukaemia is shorter than the latent period of thyroid cancers when doses on bone marrow and thyroid are equal. Another situation arises in the case when thyroid doses are much higher than doses on bone marrow. Such situation has place by the people affected by the Chernobyl accident. Tables 20 and 23 demonstrate clearly that thyroid doses among the affected populations of Belarus, Russia and the Ukraine are practically one order in magnitude higher than doses on the whole body. This is according to our point view the reason of earlier manifestation of solid cancer (thyroid cancer) than leukaemia in Belarus, Russia and the Ukraine.

## SUMMARY

An assessment of radiological consequences (stochastic effects) caused by the Chernobyl accident in Belarus, Russia and the Ukraine on the basis of experimental data on <sup>131</sup>I and <sup>137</sup>Cs in these countries has been carried out.

Results of this assessment show that the Chernobyl accident can be considered a real disaster. It can cause about 44,000 additional fatal cancers and leukaemia

among the affected populations of Belarus, Russia and the Ukraine.

On the basis of the data established in the course of this study explanations of some contradictions in manifestation of stochastic effects as a result of the Chernobyl accident have been suggested.

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