CO2-1 Production Rate of *p* Nuclei Sm-146 via Pm-146 Using Bremsstrahlung

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INTRODUCTION: The *p* process, which synthesizes the neutron-deficient nuclei, proceeds via photo-disintegration, γ process, and rapid proton capture process, rp process, on existing heavy seed nuclei in the temperature of $T_9 = 2-3 \times 10^9$ K. The high temperature can be achieved in explosive environments, such as the O/Ne layers of Type II supernovae and deflagration flame fronts in Type I supernovae. Production rate by the photo-disintegration is expressed by the product of cross section and photon flux based on Plank's black body spectrum. Thus, cross section at the vicinity of threshold is of great importance for quantitative understanding of the p process. We have performed cross section measurement of ${}^{147}Sm(\gamma,n){}^{146}Sm$ and ${}^{147}Sm(\gamma,p){}^{146}Pm$ using low energy performed bremsstrahlung, focusing on ¹⁴⁶Sm which abundance at the time solar system has been known [1].

EXPERIMENTS: Oxide of isotopically enriched samarium-147 and gold foil as a flux monitor were irradiated using bremsstrahlung which generated from bombardment of electron to a platinum plate. Activity of ¹⁴⁶Pm in the samples were measured using a HPGe detector after the irradiation. The sample, which ¹⁴⁶Pm gamma-ray peak was scarcely detected, was measured at Ogoya Underground Laboratory. Spectrum of bremsstrahlung was calculated with GEANT4 code [2].

RESULTS: Photoactivation yield of 147 Sm(γ ,p) 146 Pm reaction in the bremsstrahlung is shown in Fig. 1 together with a theoretical calculation. Product of bremsstrahlung spectrum by GEANT4 and cross section by TALYS, which incorporates Hauser-Freshbach model [3], were considered for the theoretical calculation. Photoactivation experiments with the bremsstrahlung have the limitation that the data need to be unfolded to obtain a cross section. The cross section was extracted using the product of Breit-Wigner distribution and collection factor which depends on energy. Parameters of 15 MeV for median and 4.3 MeV for natural width of were used for the Breit-Wigner distribution, based on a giant dipole resonance of A=147. The extracted cross section by the unfolding and TALYS calculation are shown in Fig. 2. As

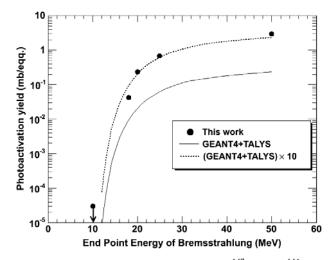


Fig. 1. Photoactivation yield of 147 Sm(γ ,p) 146 Pm reaction in bremsstrahlung.

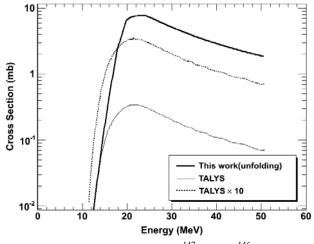


Fig. 2. Cross section of ${}^{147}Sm(\gamma,p){}^{146}Pm$ reaction.

a future plan, ¹⁴⁶Sm/¹⁴⁷Sm atom ratio will be determined with an accelerator mass spectrometry at Argonne National Laboratory [4].

REFERENCES:

- [1] A. Prinzhofer et al., Astrophys.J., 344(1989) L81-L84.
- [2] R. Brun and F. Carminati, GEANT Detector Description and Simulation Tool, CERN Program Library Long Writeup W5013 edition (CERN, Geneva, 1993).
- [3] A. J. Koning, S. Hilaire, and M. Duijvestijn, 2004, in TALYS: A nuclear reaction program (NRG-report 21297/04.62741/P) also available at http://www.talys.eu.
- [4] Kinoshita et al., J. Phys. G, **35**(2008) 014033.

採択課題番号 21009 光核反応を用いた p-プロセス核種 Sm-146 の元素合成に関する研究 共同通常 (筑波大・加速器) 木下哲一(KEK・放射線科学) 桝本和義

(東北大・核理研) 廣瀬健太郎、大槻勤(金沢大・LLRL) 浜島靖典 (京大炉) 八島浩、関本俊

CO2-2 Development of a Calibration Method for Neutron Detectors Using White Neutrons with Energy Range from Thermal to 10 keV

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INTRODUCTION: Precise determination of epithermal neutron fluence is important in various studies such as characterization of a thermal neutron reference field and irradiation dose evaluation in a boron neutron capture therapy [1]. Epithermal neutrons mixed in the thermal neutron reference field have an effect on calibration of neutron detectors. In the present study, we have developed a new calibration method for neutron detectors using a white neutron source with the epithermal energy In this calibration method, precise efficiency range. curves are experimentally determined hv two-dimensional simultaneous measurements of the pulse height of detector outputs with the flight time. We also developed an epithermal neutron spectrometer for neutrons up to 10 keV.

EXPERIMENTS: Epithermal neutrons were produced by a photo-neutron reaction using a water-cooled tantalum target at the KURRI Linac [2]. At first, a lead shadow bar was set near the target to reduce effects of "gamma flash" from the neutron source. The position of the shadow bar was adjusted and optimized by remote control. Figure 1 shows output pulses observed with a total-absorption-type BGO scintillation detector before and after setting a shadow bar. The BGO detector counts neutrons by the ${}^{10}B(n,\alpha\gamma)$ reaction and the reference neutron spectrum from the neutron source was obtained in a energy range up to 10 keV by the time-of-flight (TOF) method. Fast neutrons with energy above 10 keV from the neutron source can disturb calibration focusing on the epithermal region for detectors with a neutron moderator, because the fast neutrons appear in epithermal neutron region on the TOF spectrum due to a slowing-down time of a few microseconds in the moderator. A 2"×2" NE213 liquid scintillation detector was used to measure the fast neutrons with the TOF method.

The epithermal neutron spectrometer is composed of a ³He proportional counter, a polyethylene moderator and filters made of Ag, Co, W, Mo and Mn having several absorption resonances. Response functions of the spectrometer were measured by the TOF method. The epithermal neutron spectrum was determined by an unfolding method using the response functions.

RESULTS: Figure 2 shows the TOF spectrum measured with the liquid scintillation detector, indicating that

the gamma flash component is rejected by the pulse shape discrimination with the liquid scintillation detector. Linac-produced noise signals were found on the TOF spectrum which should be suppressed by improving the signal processing method.

The epithermal neutron spectrometer gave the response functions in Figure 3(a) and the epithermal neutron spectrum in Figure 3(b) obtained by being unfolded with SAND-II code. Figure 3(b) also shows the neutron spectrum obtained from the TOF measurement with the BGO detector. The two spectra are in good agreement except in the thermal energy region.

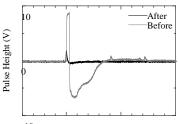


Fig. 1. Output pulses observed with the BGO detector before and after setting the shadow bar.

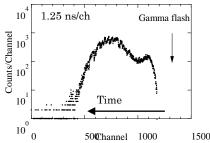


Fig. 2. TOF spectrum measured with the liquid scintillation detector.

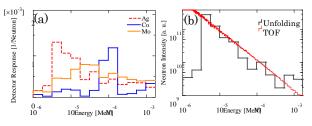


Fig. 3. (a) Response functions of the epithermal spectrometer for the Ag, Co and Mo filters, (b) epithermal neutron spectra obtained with the unfoilding method and the TOF method.

REFERENCES:

Y. Sakurai and T. Kobayashi, Nucl. Instr. Meth., A453 (2000) 569-596.K. Okano *et al.*, Nucl. Instr. and Meth., 186 (1981) 115-120.

[2] K. Kobayashi et al., Annu. Rep. Res. Reactor inst. Kyoto Univ. 22, 142 (1989)

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